

CCA (Coated Conductor for Applications) 2025 workshop

HTS fusion technology status in China

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Geneva, Switzerland

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Institute of Plasma Physics, Chinese Academy of Sciences

Outline

1 Background of HTS technology for Fusion

2 HTS technology development in China

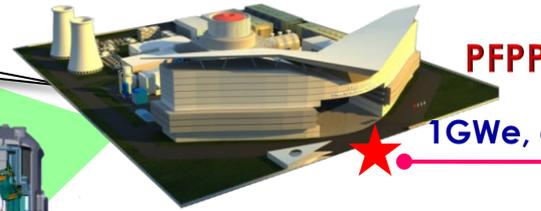
3 HTS fusion device status in China

4 Summary

Background of HTS technology for Fusion

1-2GW, 10-20 years engineering life, Secure, stable and tritium self-sustaining

Steady-state long-pulse discharge, tritium breeding, dpa ~20, annual consumption of tritium ~6-10kg



PFPP

1GWe, electricity generating station

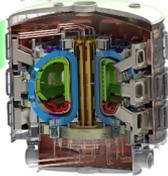


CFETR

- I: $Q=1-5$, steady-state, $TBR>1$, $>200MW$, 10dpa**
- II: DEMO validation, $Q>10$, CW, 1GW, 50dpa**

50~100MW, $Q>1$

Demonstration of power generation



BEST

Phase I: $Q=10$, 400s, 500MW, Hybrid burning plasma

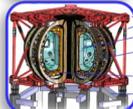
Phase II: $Q=5$, 3000s, 350MW, Steady-state burning plasma



ITER



EAST **Advanced PFC, advanced steady-state operation mode**



HL-2M **Advanced divertor, High power H&CD**



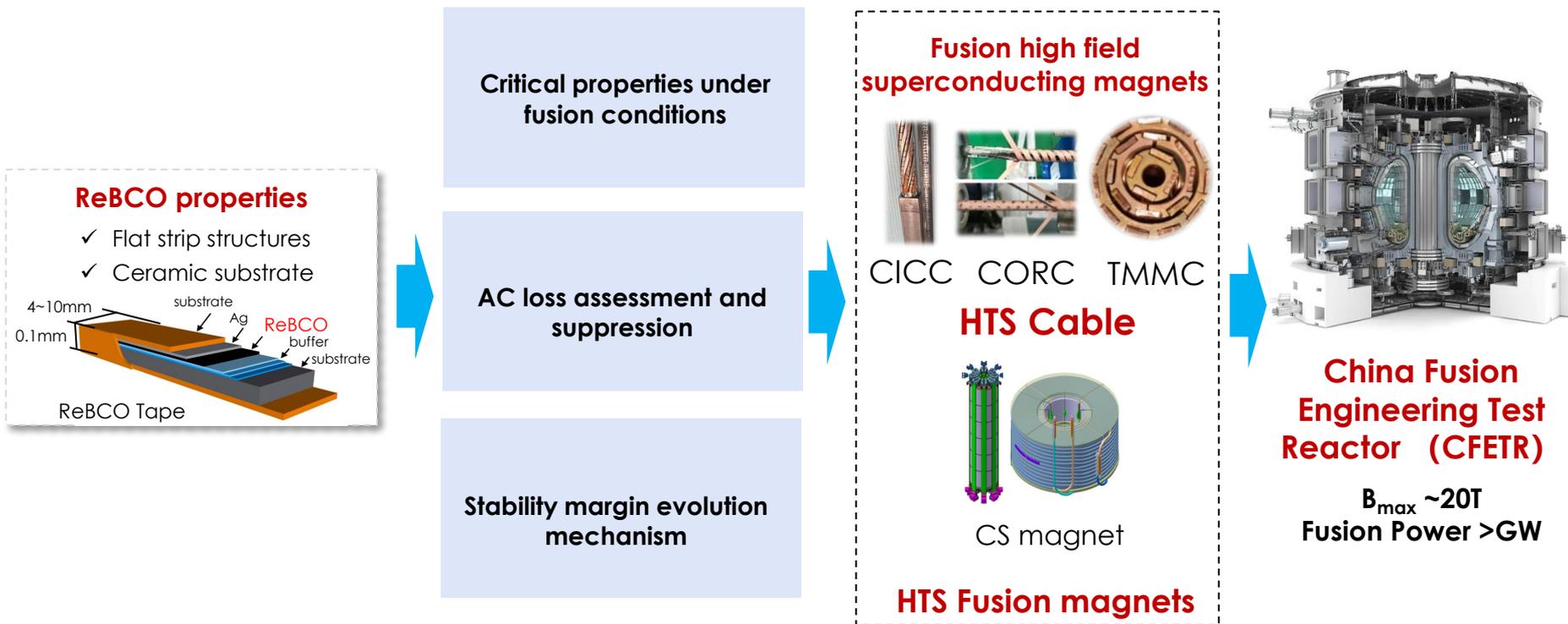
J-TEXT **Basic plasma**

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- 2 HTS technology development in China**
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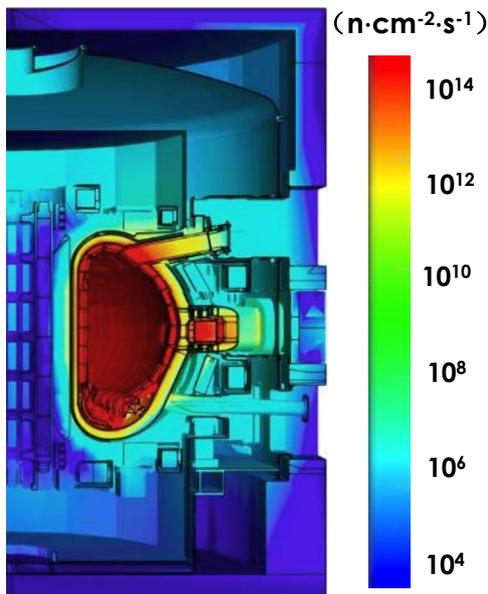
HTS technology development

- **Key challenges for safe operation of fusion HTS magnets**
 - Critical performance, AC loss, Stability and Engineering preparation

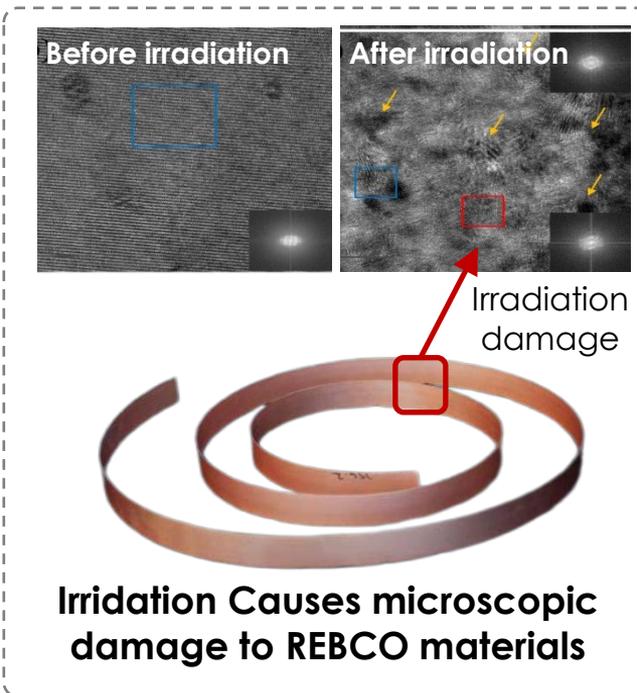


HTS Technology: (1) Critical Performance under Fusion Irradiation

- Neutron radiation produced by the **Fusion D-T reaction** will accumulate in the superconducting magnet, **causing critical performance degradation and affecting the safe operation**



Fusion flux density distribution



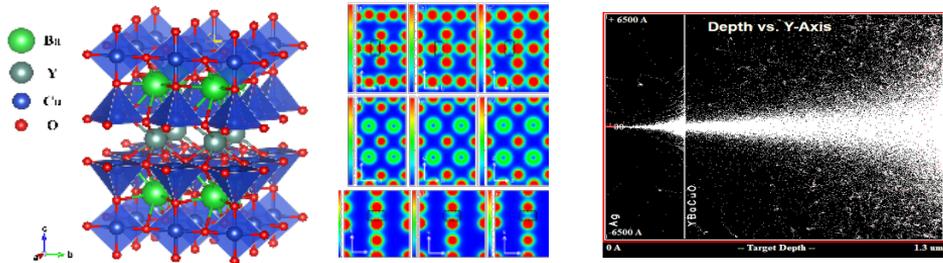
TF Magnet peak cumulative dose

$1.41 \times 10^6 \text{ Gy} @ 2.5 \times 10^{25}$

HTS Technology: (1) Critical Performance under Fusion Irradiation

- **Irradiation simulation of REBCO: based on first principles and molecular dynamics simulations.**
 - with appropriate irradiation, oxygen vacancies and lattice distortion are induced, forming effective pinning centers and improving J_c .
- **Irradiation experiments of REBCO: Proton/Gamma-ray/ Neutron irradiation**

Theoretical analysis of irradiation damage in HTS tapes

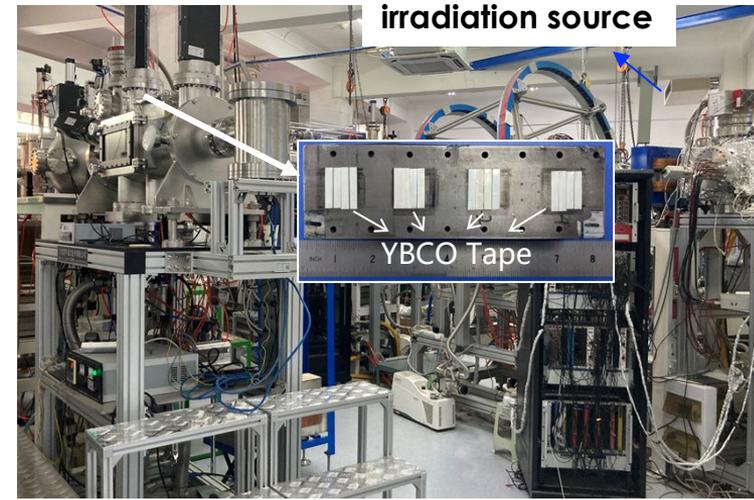


Structural model of YBCO

Particle incidence depth simulation

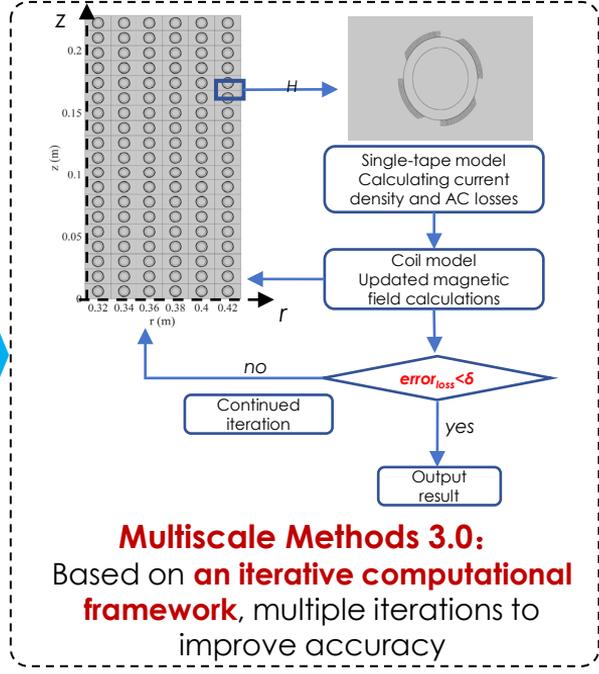
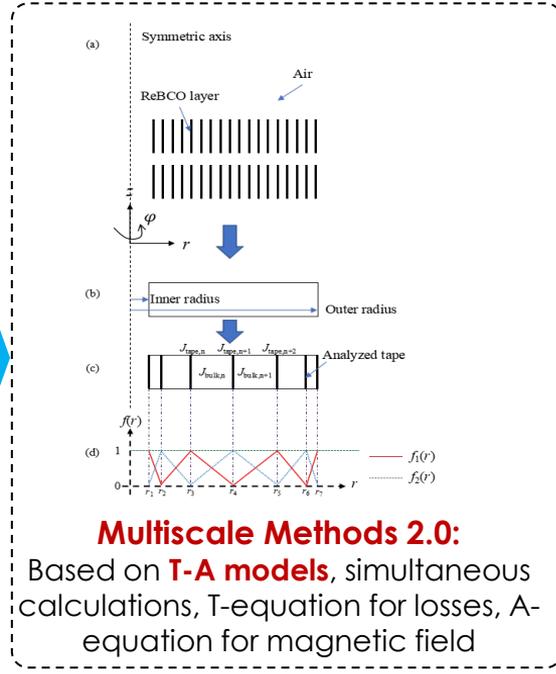
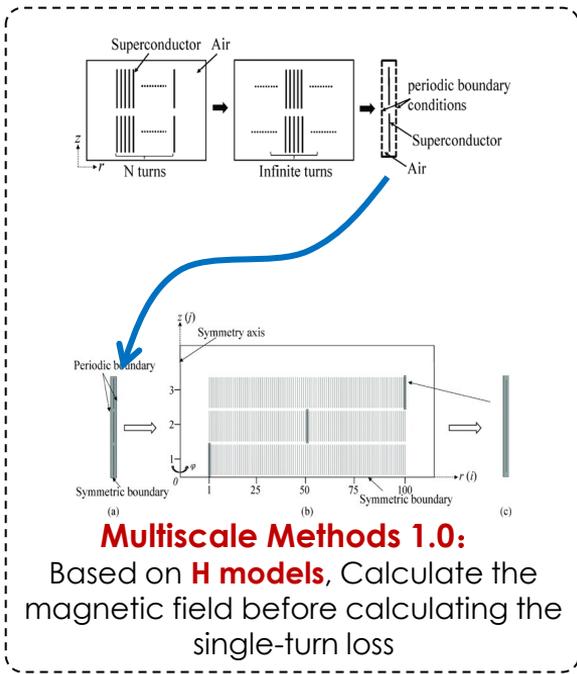
Irradiation fluence	Thickness of YBCO layer	Radiation damage (DPA)
5.0×10^{14} p/cm ²	1.5 μ m	2.85×10^{-4}
1.0×10^{15} p/cm ²	1.5 μ m	5.70×10^{-3}
2.0×10^{15} p/cm ²	1.5 μ m	1.14×10^{-2}

Irradiation Experiment on HTS tapes



HTS Technology: (2) AC loss calculation Model

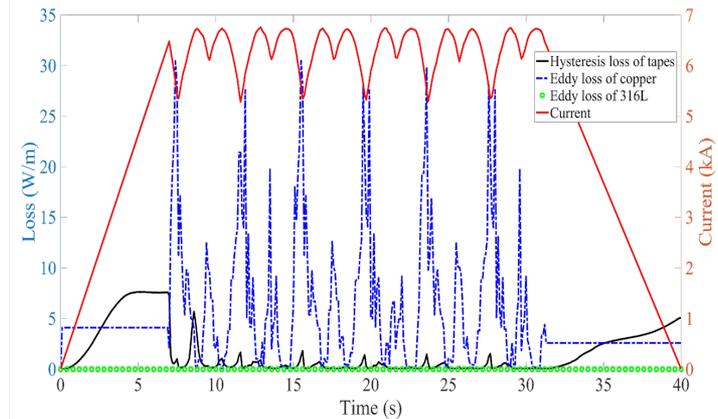
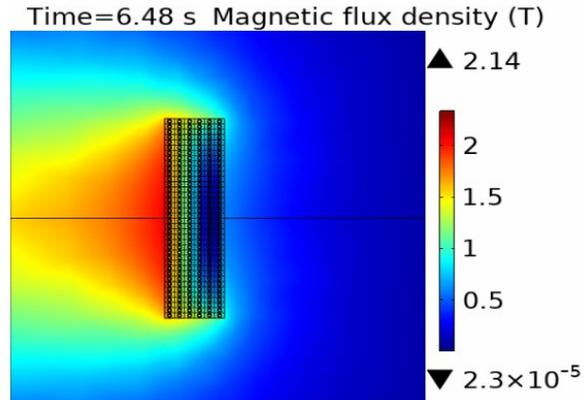
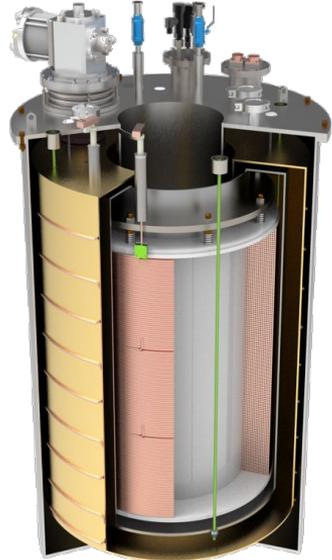
- **3 versions of multi-scale AC loss calculation method:** to solve the problems of time-consuming and non-convergence of the AC loss calculation of large-scale HTS magnets



HTS Technology: (2) AC loss calculation Model

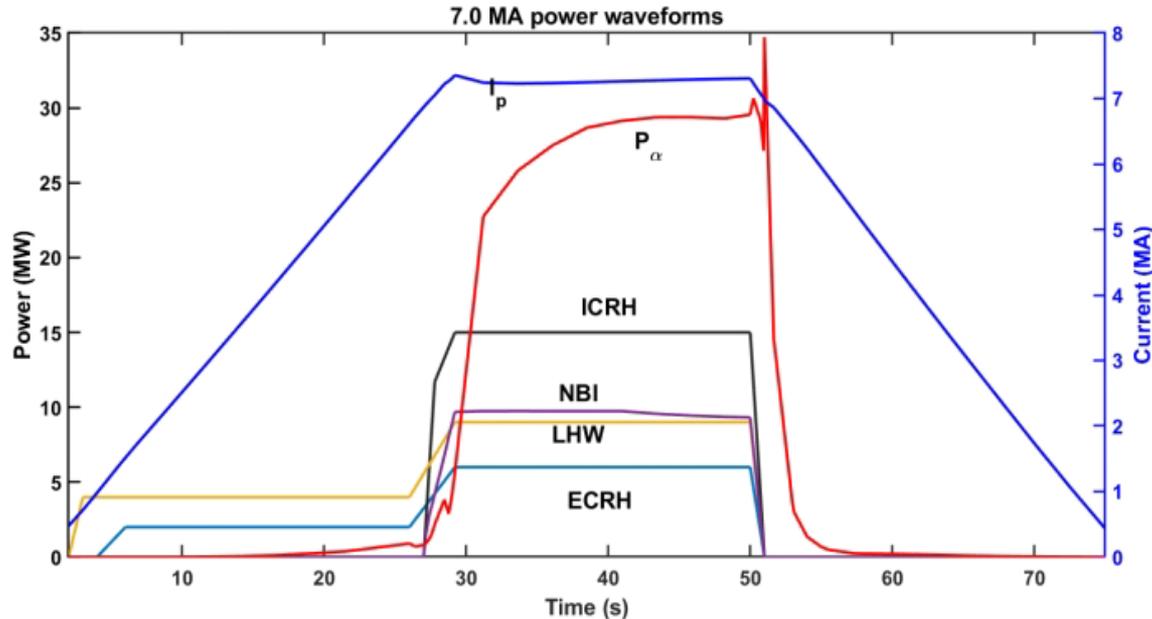
- **Applications and advantages of multiscale methods:** can be applied to both 2D and 3D calculations, dramatically improving computational efficiency with high accuracy

- ✓ **AC loss calculation of solenoid coil prepared with HTS spiral-wound conductor (like CORC)**
- ✓ **Compared with other computational methods:** Higher solution accuracy and faster computation time



HTS Technology: (3) Stability Analysis Model

- Systematic stability model is established, it can realize accurate **AC loss and Nuclear heat loading, achieving 0.01 K high-precision temperature margin calculation**



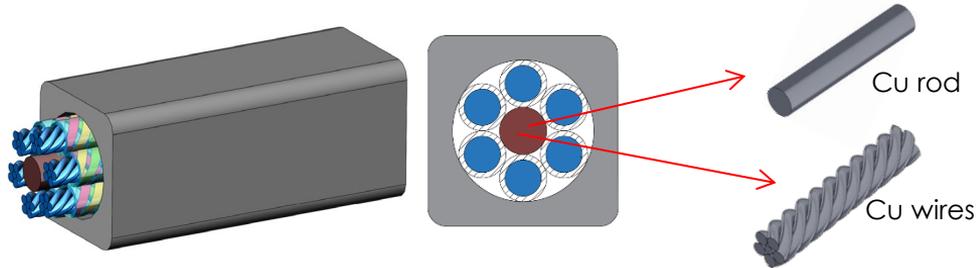
Nuclear Heat of BEST Fusion device with 130 MA Fusion Power

Technical Difficulties:

- ❑ Dynamic AC loss and Nuclear Heat Deposition
- ❑ Location to calculate minimum Temperature Margin
- ❑ 0.01 K ΔT accuracy and 1 mm position accuracy

HTS Technology: (3) Stability Analysis Model

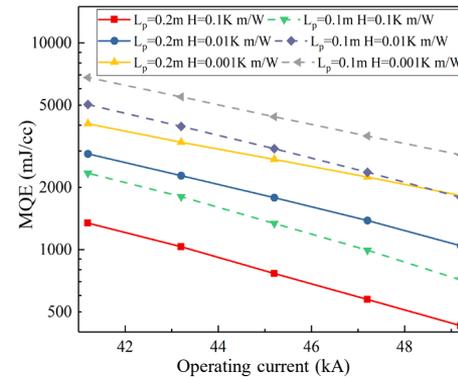
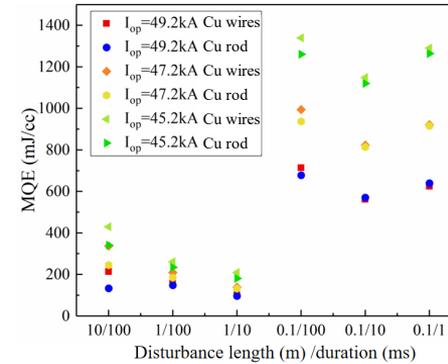
- Established stability analysis model is used to calculate temperature margin of ReBCO conductors for fusion reactor high-field environments



Schematic of REBCO conductor and Different Centre Skeleton (Cu wires and Cu rod)

The main parameter of REBCO conductor

Maximum operating current (kA)	49.2
Peak magnetic field (T)	14.32
Operation temperature (K)	4.2
CORC diameter (mm)	10.4
Cable diameter (mm)	32
Centre Skeleton Diameter (mm)	8



The MQE calculations for the conductors were completed quickly and accurately.

HTS Technology: (4) HTS Conductor

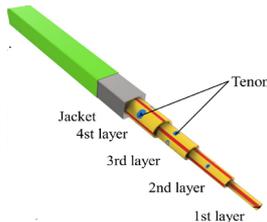
● HTS conductors are the only option for realizing 15 T to 20 T class magnetic fields

✓ Many kinds of HTS conductor structures have been proposed in China

Institute of Plasma Physics,
Chinese Academy of Sciences

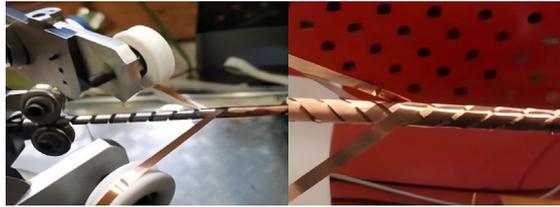


CORC with variable winding
angles, **10kA@10K, self-field**



The Tenon-mortise-based modularized
conductor (TMMC), **10.8kA@77K, SF**

Hefei International Applied
Superconductivity Center and
Institute of Plasma Physics

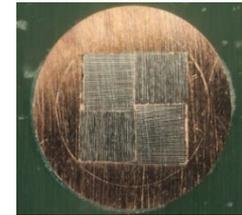


Highly Flexible ReBCO Cable
I_c~10.6kA @4.2K, 20T

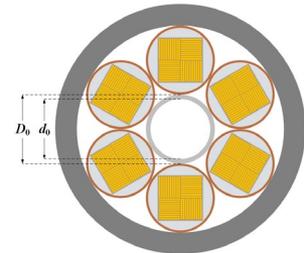


CORC-CICC, **6kA@4.2K, 21T**

North China Electric
Power University



Quasi-isotropic conductors,
2.3 kA@77K, self-field

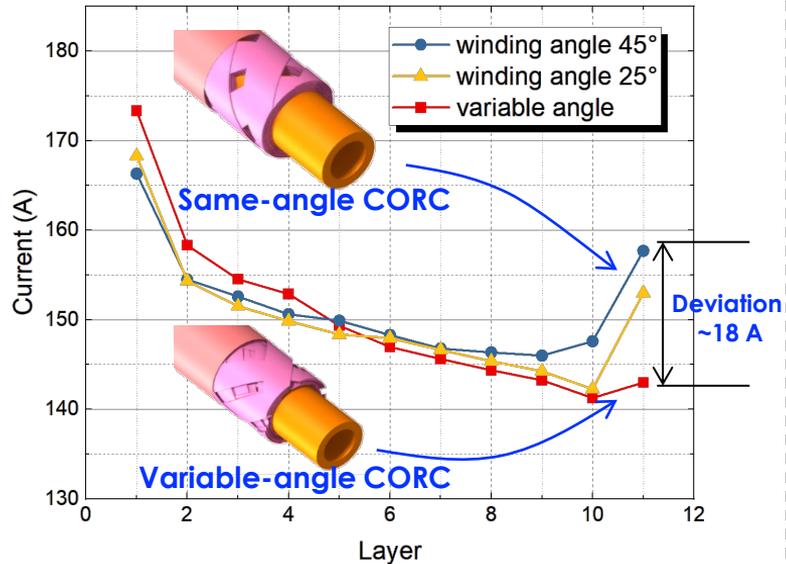


Q-IS-CICC
20 kA@77K, 0.5T

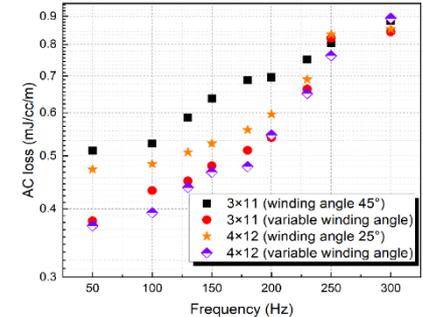
HTS Technology: (4) HTS Conductor (variable-angle CORC)

- The **variable-angle CORC** optimise current distribution and reduce the current sharing of the outer tapes by adjusting the winding angle of each tape layer, thus significantly reducing overall losses

- ✓ The reduction of current flowing through the outermost tapes.



- ✓ The CORC cable with variable winding angles achieves **31% loss reduction**

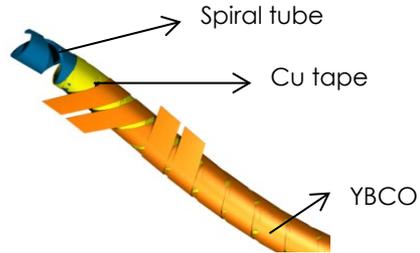


AC losses of four structural conductors at different frequencies (50 mT)

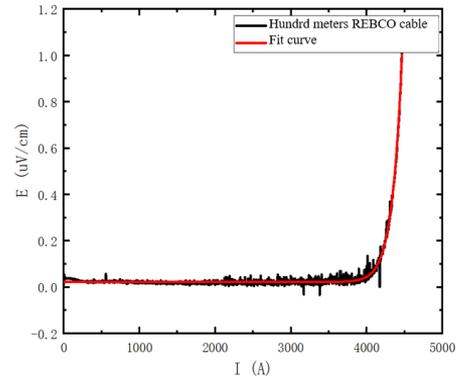
Winding angle	layers	Ac loss	Reduction
45°	3×11	0.41mJ/cycle/m	31%
variable	3×11	0.28 mJ/cycle/m	

HTS Technology: (4) HTS Conductor (HFRC and CORC-COCC)

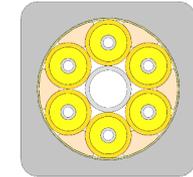
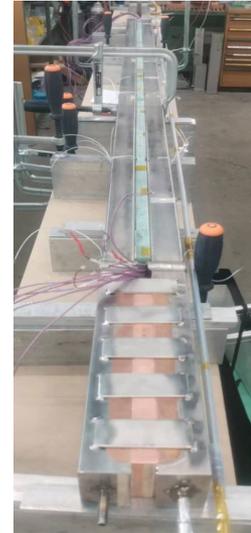
- **Highly Flexible REBCO Cable (HFRC)** : use **spiral tube** as the mandrel to wind the REBCO tapes
- **Full size CORC-CICC conductor sample** have been manufactured and tested at Sultan lab



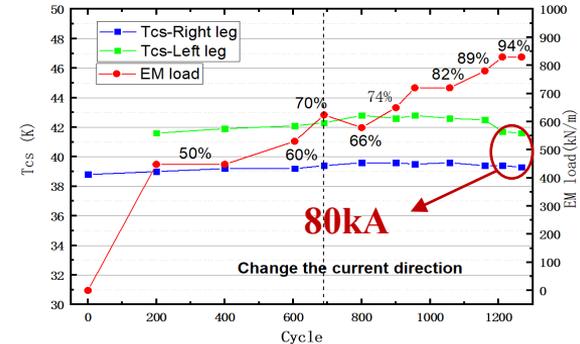
Highly Flexible REBCO Cable (HFRC)



Hundred meters cable
 $I_c \sim 4468 \text{ A}$, $n=34.1@77\text{K}$, SF



CORC-CICC

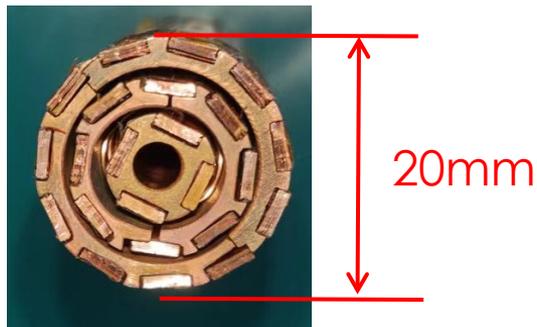


Tests carried out for performance verification

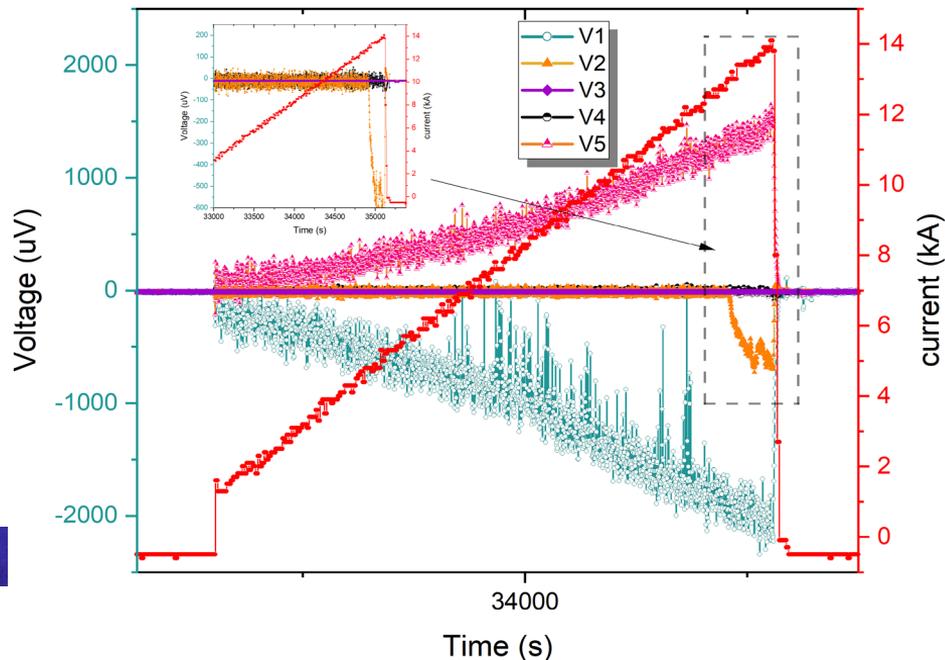
- ✓ 1266 EM cycles
- ✓ 2 WUCD (RT-4.5K) cycles
- ✓ Quench campaigns @ **47kA, 55 kA, 60kA&65kA**

HTS Technology: (4) HTS Conductor (New TMMC)

- The **TMMC conductor** is manufactured and tested at 77K, **$I_c=13.69$ kA@77K,SF**. Engineering critical current density is **18.1 A/mm²**



3-layer TMMC conductor

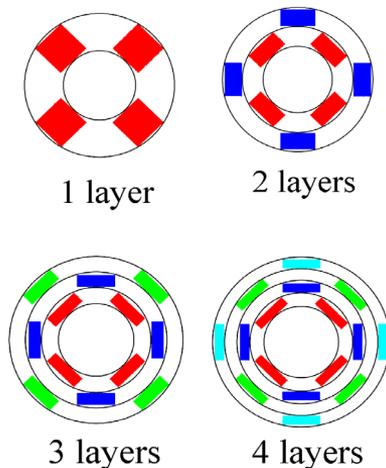
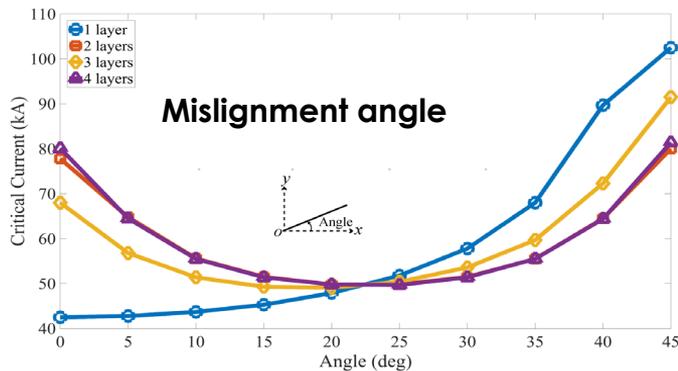
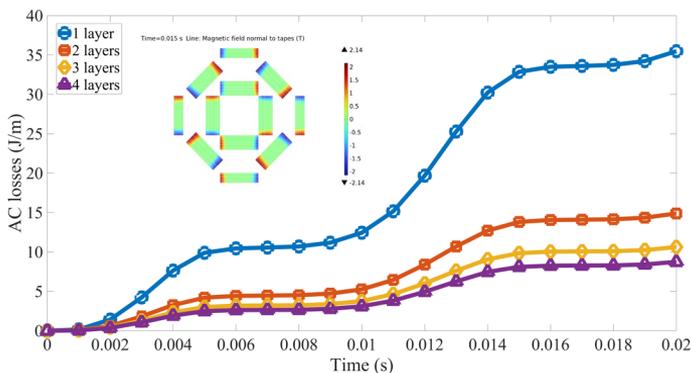


**I_c Test of TMMC conductor with liquid nitrogen
13.69 kA@77K, Self-field**

HTS Technology: (4) HTS Conductor (New TMMC)

● Design and Preparation process optimization to improve Engineering J_c of TMMC

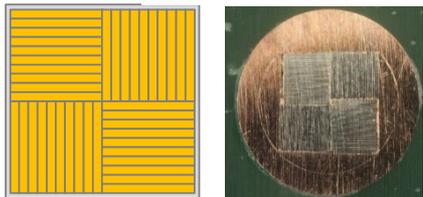
- ✓ Change the number of layers and mislignment angle to reduce AC loss
- ✓ Reduce the thickness of former and increase the density of HTS stacks



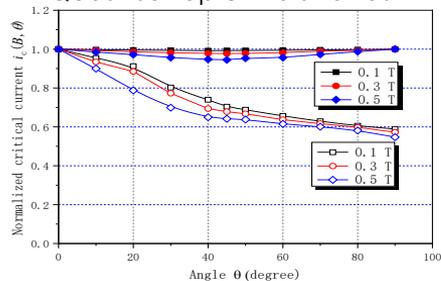
HTS Technology: (4) HTS Conductor (Quasi-isotropic HTS Strands)

● Development of CICC made from Quasi-isotropic HTS Strands

Q-IS

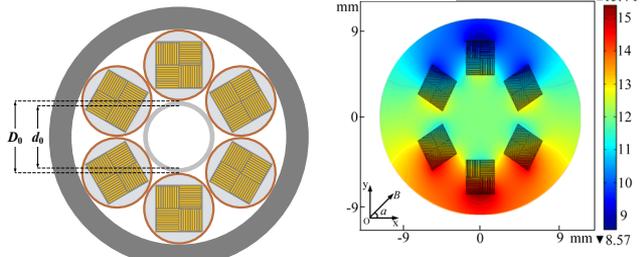


Quasi-isotropic HTS Strands



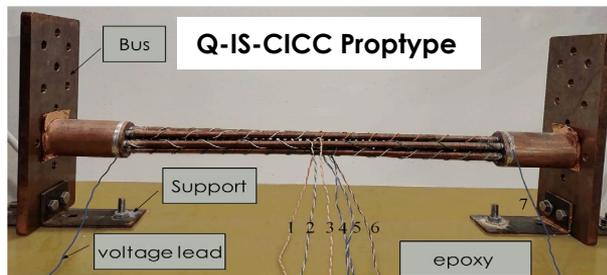
Better critical characteristics at different angles of the external magnetic field

Q-IS-CICC



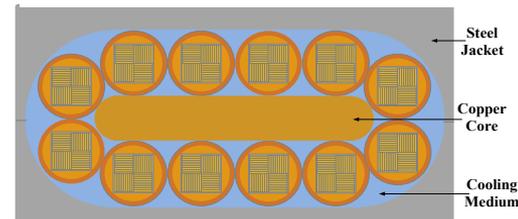
Q-IS-CICC

$\theta=0^\circ$ with transport
236.76kA@12 T

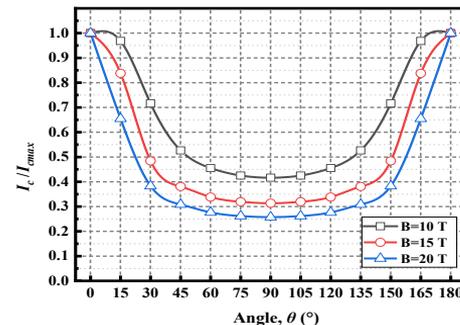


Tested $I_c=12.5kA@1.5T, 77K$

Q-IS-Rutherford Cable



Q-IS-Rutherford Cable



Designed Critical current at 4.2K

#This work is from North China Electric Power University

Cheng Junhua, et al, IEEE Trans. Appl. Supercond.,35(5): 4800608(8pp), 2025.
He Ye, et al, IEEE Trans. Appl. Supercond. 35(5): 4800207(7pp), 2025.

Outline

- 1 Background of HTS technology for Fusion
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- 4 Summary

China Fusion Engineering Test Reactor (CFETR)

Participation in ITER

1. PF6 Magnet manufacture
2. TF conductor development
3. 100% CC coil development
4. 100% Feeder development
5. CC & Feeder Conductors
6. TF magnet support
7. Power supply system

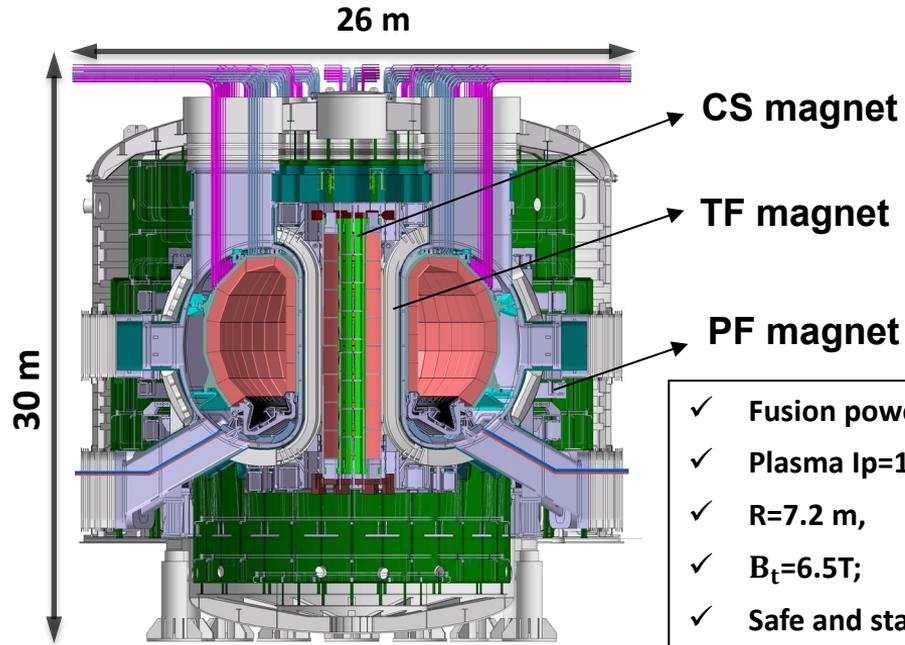
CFETR Engineering design

- 1.3 years conceptual design (2012~2014)
- 2.7 years engineering design (2015~2021)

CRAFT construction

1. HTS conductor
2. TF prototype manufacture
3. CSMC and testing facility
4. SC conductor testing facility
5. SC material testing facility
6. SC magnet testing facility

- ❑ Explore and master fusion DEMO level key technologies
- ❑ Establish the method and standard for CFETR development



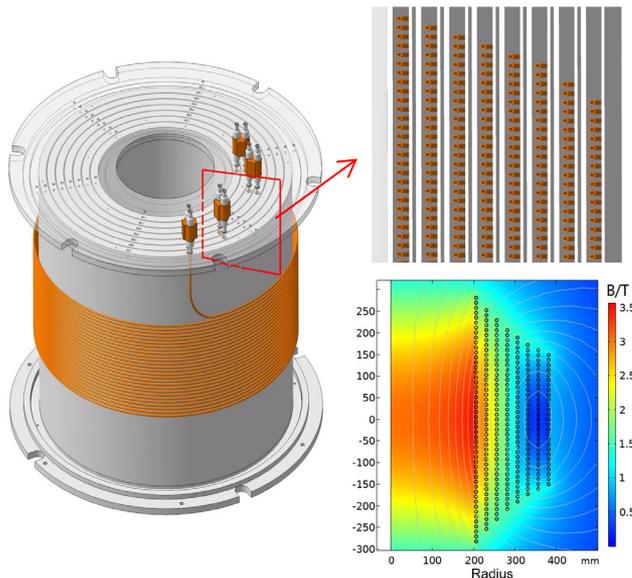
China Fusion Engineering Test Reactor

- ✓ Fusion power ~ 1000 MW
- ✓ Plasma $I_p=11\sim15\text{MA}$,
- ✓ $R=7.2\text{ m}$,
- ✓ $B_t=6.5\text{T}$;
- ✓ Safe and stable, realize long-pulse steady-state operation;

Development of HTS Solenoid Coil in ASIPP

● High current capacity HTS solenoid coil with forced-flow cooling

- ✓ 8 sub-solenoid coils assembled coaxially and connected in series.
- ✓ It is cooled by forced-flowing cooling with high cooling efficiency.



The conductor arrangement inside solenoid coil and magnetic field distribution

Main parameters of solenoid coils	
Parameter	Value
Operating current	6 kA
Layers in radial direction	8
Height of coil	985 mm
Inner/Outer diameter of coil	194.5mm/415.5mm
Total length of tapes	20.58 km
Inductance	28.21 mH
Cooling	Forced-flow with supercritical helium

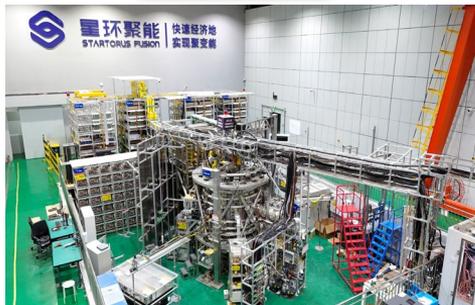


Solenoid coil

Startorus Fusion HTS Fusion Device

- **Startorus Fusion** is located in Xi'an, China, and is dedicated to the commercial application of fusion energy and the development of related technologies.

2022-2024

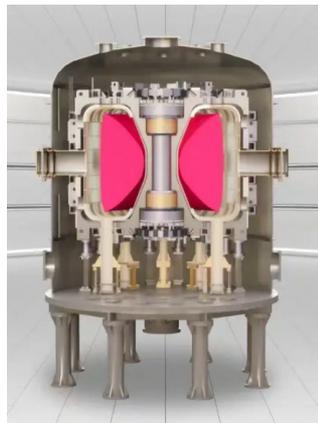


SUNIST-2: copper
Startorus Fusion &
Tsinghua University

Initial engineering
validation

2024-2025

NTST: LN cooled copper Startorus Fusion



Current Position

$$0.8 < \delta < 0.8$$

$$R_0: 0.72 \text{ m}$$

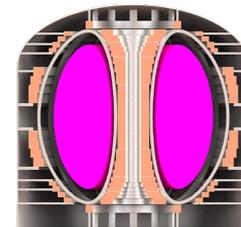
$$\alpha: 0.4 \text{ m}$$

$$\kappa: 2.2$$

$$B_T: 1 \text{ T}$$

Negative triangularity exploration
and cryogenic operation

2024-2027



CTRFR-1: HTS
Startorus Fusion

Full engineering verification

2027-2030



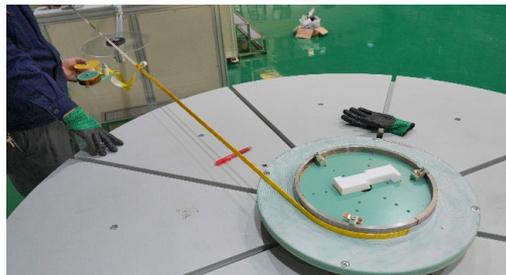
CTRFR-2: HTS
Startorus Fusion

Demonstration fusion power plant

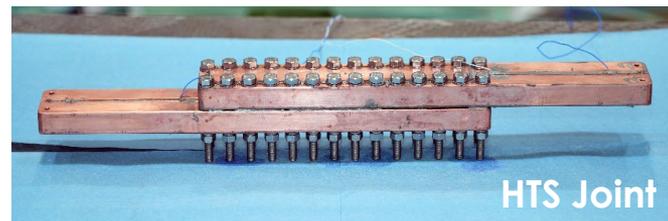
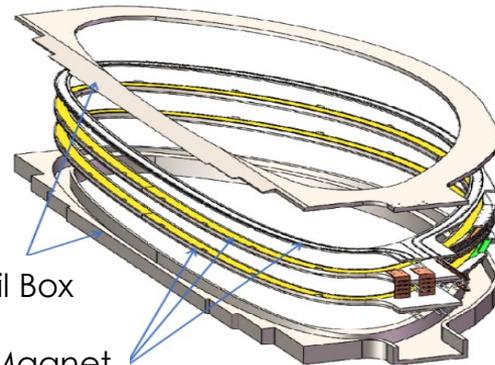
Startorus Fusion HTS Fusion Device

● R&D of HTS magnets

- ✓ 3 T D-shaped TF magnets for next step spherical tokamak
- ✓ Ultra compact CS magnets for next step spherical tokamak



Compact CS magnets

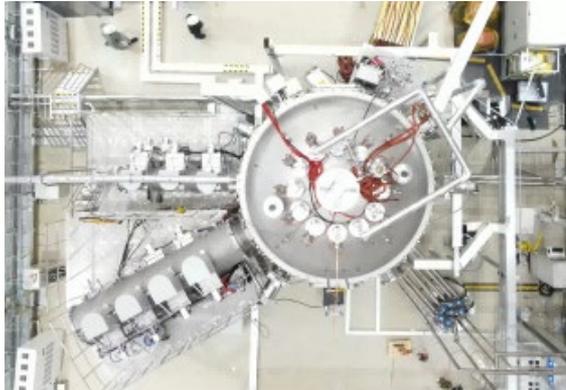


D-shaped TF magnets

Energy singularity HTS Fusion Device

- **Energy Singularity** built and operated first all HTS tokamak—HH70

✓ HH70 was built and operated in 2024, all-HTS tokamak, validated and demonstrated HTS application in fusion device.



HH70

R_0 0.75m

B_0 1T, B_{max} 3.1T

the world's first HTS tokamak



Jingtian magnet

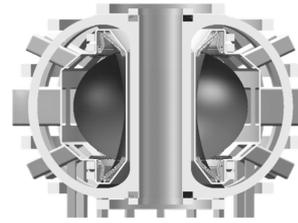
$B_{coil} \geq 25T$

$J_e \geq 180MA/m^2$

to retire risk for HH170

TF magnet

2025 estimate



HH170

R_0 1.65m

B_0 10T, B_{max} 25T

the most economic tokamak

to achieve $Q_{DT}^{eq} \geq 10$

2027 estimate



HH380 D-D Reactor

R_0 3.8m, B_0 12T, B_{max} 29T

$P_{electric}$ 500MWe, P_{fusion} 1.3GW

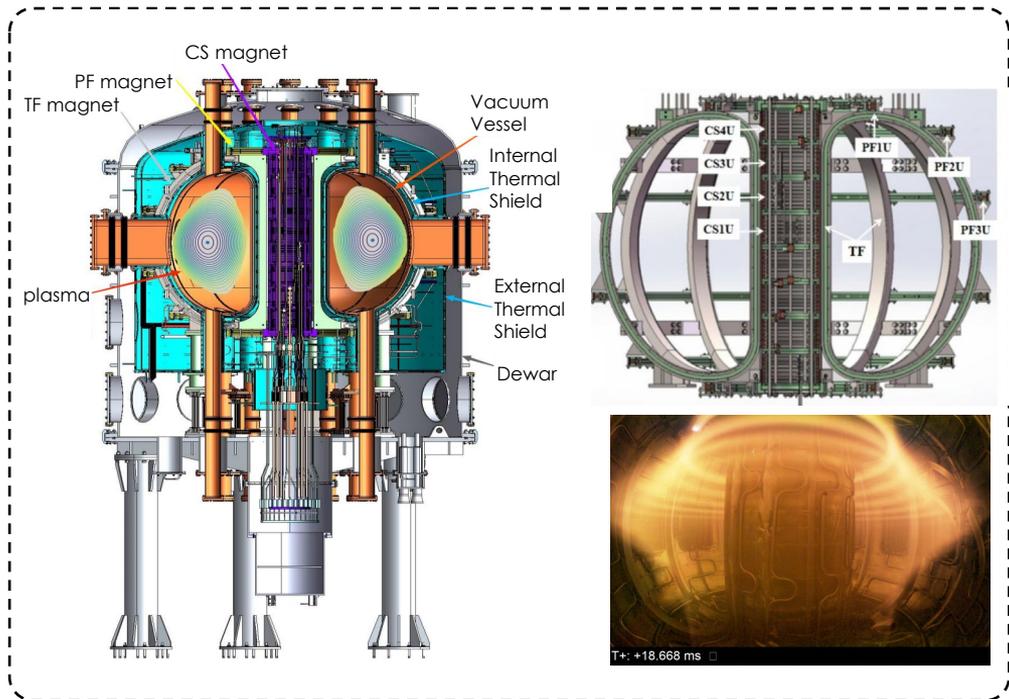
to demonstrate

electricity production

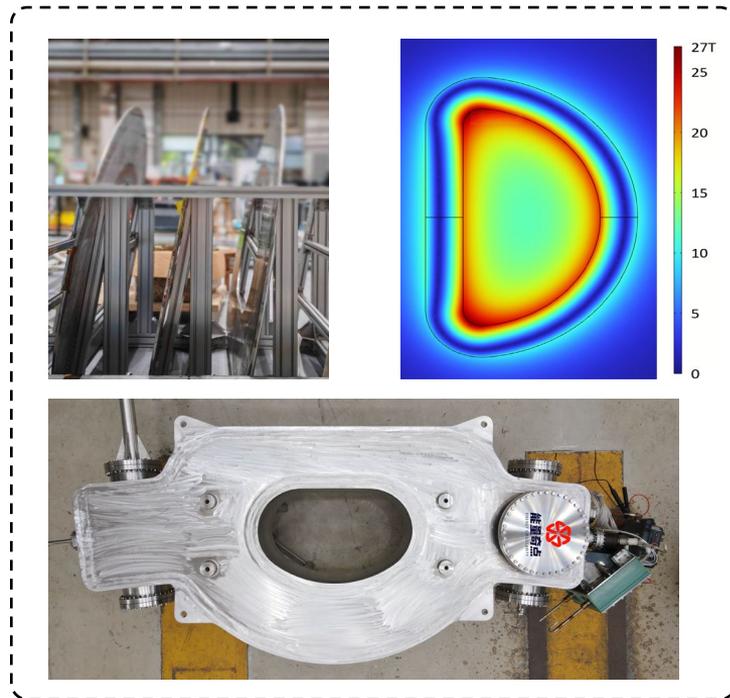
early 2030s

Energy singularity HTS Fusion Device

- HH70 supports the next generation HH170 R&D of the Energy Singularity
- “Jingtian” magnets are currently being developed with a target: $B_{max} \geq 25T$



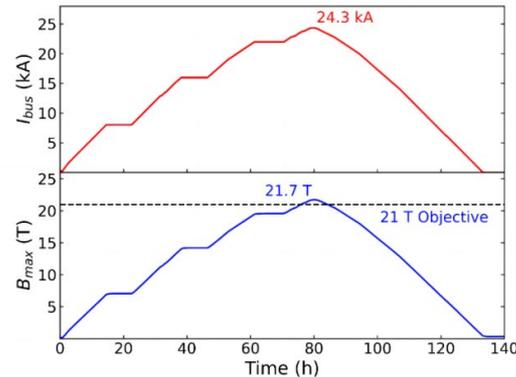
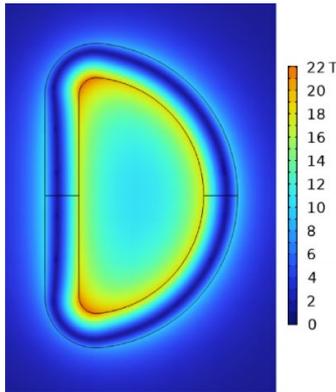
HH70 HTS Tokamak



Jingtian HTS magnet

Energy singularity HTS Fusion Device

- “Jingtian” magnets has generated a magnetic field of up to **21.7 T** in the first round of through-flow experiments, **setting a record for the highest magnetic field in a large-aperture, HTS D-shaped magnet.**



“Jingtian” magnet was energized with a current of **24,300 A**. The total number of ampere-turns in the magnet reached **9,260,000 a-t**, and the J_{ce} of the winding reached **157 million A/m²**.

Other HTS Application: (1) Superconducting motor

- Based on fusion superconducting magnet technology, successfully developed **HTS motor in aircraft propulsion** (98% efficiency, heat leakage <5W)
- Assembled with the UAV "LQ-H" and accomplished **flight verification**



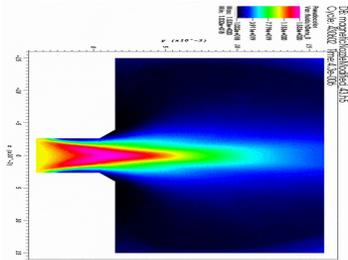
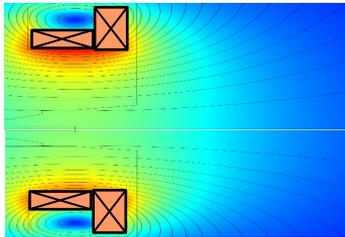
Flight verification (Flight altitude ~500m, Total weight: ~150kg)

Zheng J X*, et al., Flight verification of cooling self-sustaining high-temperature superconducting motor, *Superconductor Science Technology* 37 07LT02, 2024

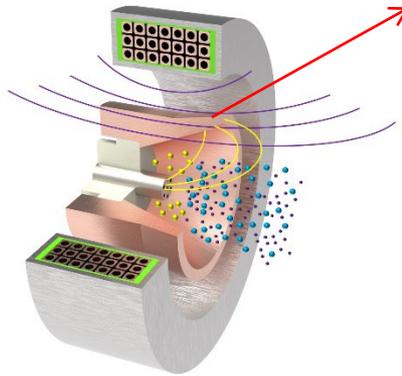
Sun JX, Zheng JX*, et al., Design of Axial Flux HTS Machine Prototype and AC Losses Calculation, *IEEE Transactions on Applied Superconductivity*, 33(8), 5203906, 2023

Other HTS Application: (2) Superconducting MPDT

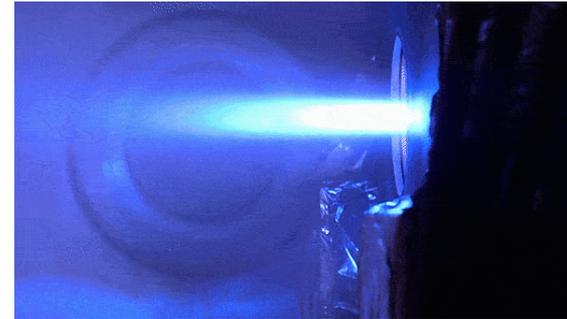
- Based on the high precision gradient field superconducting magnetic potential control technology, the 100 kW SC magnetoplasmadynamic thruster (MPDT) has been successfully developed



Gradient field magnetic potential optimization



Superconducting magnet



- Specific impulse > 6000s
- Thrust > 5.0N
- Efficiency > 70%

Aftab H, Zheng JX*, et al., Exploring efficiency in next generation high temperature superconducting-enhanced applied field magnetoplasmadynamic thrusters: A combined numerical and experimental study, *Acta Astronautica* 223 448-61, 2024

Liu HY, Zheng JX*, et al., Research on the Gradient-Field Superconducting Magnet for Magnetoplasmadynamic Thruster Performance Improvement, *IEEE TPS*, 50(12), 2022

Zheng JX*, et al, Integrated Study on the Comprehensive Magnetic-Field Configuration Performance in the 150 kW Superconducting Magnetoplasmadynamic Thruster, Scientific Report 11, 20706, 2021

Outline

1 Background of HTS technology for Fusion

2 HTS technology development in China

3 HTS fusion device status in China

4 Summary

Summary

- **HTS technology is the key to construction of the next generation of compact fusion reactors**
- **The research on the technology of HTS fusion device has been fully carried out in China. The research on the technology of high-field HTS magnet has just been initially established.**
- **ASIPP has carried out a series of research works on critical performance, AC loss, Stability analysis and engineering preparation of HTS conductors. High-field HTS insert magnet and fusion magnet above 20 T are being developed in ASIPP.**
- **Several companies in China (Startorus Fusion & Energy singularity and so on) have established small-scale HTS fusion experimental devices. The fusion device with a higher magnetic field will be built in the next step.**

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Thank you for your attention!

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